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fuel assembly and support grid structure. The TMI-2 defueling operations are made difficult because, unlike most other underwater experiences, the required activities in the reactor vessel cannot be reduced to a set of precisely defined problems. The unknown conditions beneath the upper core layer at TMI-2 during the tooling design phase imposed a basic requirement to provide a set of tools with maximum versatility and usefulness for a wide range of conditions that could not be precisely defined.

UNDERWATER TOOLING DESIGN

The basic tool kit designed for early core removal activities includes long-handled tools that are either manually or hydraulically operated for the purposes of picking up and placing into fuel canisters fuel rods or pieces of control rods and fuel assembly upper end fittings that lay on the rubble bed after they were dislodged from the upper plenum grid structure. A special underwater vacuum system is also designed to clear the loose rubble that lay under the end fittings.

Once the upper layers are cleared of loose debris, the major problems associated with breaking up and removing the anticipated hard-crust material of liquefied fuel and cladding will have to be resolved with more elaborate tooling. Unknown conditions in the lower core region, which was assumed to include partial fuel assemblies possibly fused with once-molten core material, will also require special tooling. A summary of the tooling that is designed for this unique core removal operation is discussed below.

A manual tool positioner provides a strongback capable of positioning various defueling tools in all areas of the reactor vessel. It provides four degrees of freedom for the defueling tools: vertical travel, lateral positioning, rotation around the main axis, and rotation around the lateral positioner axis. A hydraulically operated remote manipulator arm with six additional degrees of freedom can be installed on the tool positioner and used to operate cutting equipment, to help guide long-handled tools or underwater video cameras, and to pick and place core debris.

Abrasive/waterjet cutting equipment will be used to break up the hard crust layer of the core, to cut fuel debris and fuel assemblies into pieces small enough to fit into fuel canisters, and to cut access holes, if required, in lower vessel components (e.g., lower grid, flow distributor plate, etc.). A hydraulically operated abrasive saw will also be used to cut the anticipated hard crust layer of the core, and to cut fuel debris and fuel assemblies into pieces small enough to fit into fuel canisters. A hydraulically operated impact chisel will be used to break up the hard crust layer of the core. A cutting station will be used to clamp and cut ductile pieces that are too large to be loaded into fuel canisters, but could be more easily sectioned in a stationary position than on the core surface. Cutting will be accomplished with a hydraulically operated wire saw.

SUMMARY OF TMI-2 DEFUELING EXPERIENCE

Defueling of the TMI-2 reactor core commenced on October 30, 1985. Initial efforts included knocking down a few partially intact fuel assemblies that were restricting full use of the defueling system from the rotating work platform. Many of the upper fuel assembly end fittings were distorted or fused together and had fuel rod stubs and control rod spider components attached. These pieces were too large to be directly placed into fuel canisters and had to be supported in the debris cutting station and properly sized using cutting shears and other tools. The first few canisters were then loaded with these partial fuel assemblies and upper end fittings along with miscellaneous control rod spiders and fuel rods.

Early efforts to pick-and-place material into the fuel canisters were not very efficient because of the maze of partial-length fuel assemblies and other rod-like structures in the upper core region and the relative bulkiness of the special tool-

ing, which was being manipulated from 8 m above the target debris pieces. Most of the tools require direct vertical access for greatest effectiveness, and any overlying material would inhibit tooling maneuverability. Using the large cutting shears, most of the upper interfering pieces of the damaged core were cut away and improved access was gained to the underlying material. These pieces were then also loaded into defueling canisters.

Defueling of the upper core region will continue with vacuuming of the loose, pellet sized and smaller debris and removal of as many loose partial fuel assemblies as possible. A U.S. Department of Energy-sponsored core stratification survey will be made by taking several core bore samples of the underlying mass, to assess its strength and composition. Various configurations of special tooling described above will be used to then break up and remove the material in the lower regions, which is scheduled to occur through the middle of 1986.

SUMMARY

By the middle of 1986, it is expected that the experience gained from these TMI-2 underwater operations will advance the knowledge of performing complex operations in support of defueling and maintenance in nuclear plant reactor vessel and storage pools.

4. Instrument Sensing Line Anomalies in BWRs, J. S. Miller (*Gulf States Util, St. Francisville*), J. C. Elliott (*GE, St. Francisville*)

This paper describes the instrument sensing line anomalies found at the River Bend Station (RBS), approaches that were used to identify solutions, and modifications that were used to bring the instrument sensing lines within design specifications. Also, instrument sensing lines at other reactor facilities that exhibit the same symptoms as those experienced at RBS will be discussed.

DISCUSSION

When the RBS reactor was first pressurized, it was observed that several leak detection instruments that measure steam flow in the reactor core isolation cooling (RCIC) steam line indicated erratically. One of the four flow measurement instruments (all on the same steam line) indicated a substantial steam flow when the RCIC turbine was not running. Another instrument indicated a slightly negative reading when there was no flow, then drifted strongly negative when steam flow was established, and then slowly returned to initial indication after steam flow stopped.

The purpose of these instruments is to detect excessive steam flow indicative of a steam leak or line break, and to initiate an automatic closure of isolation valves. Since these instruments are safety-related, they must satisfy technical specifications established by the U.S. Nuclear Regulatory Commission. These specifications require that the instrumentation satisfy accuracy and operability requirements, or that the affected isolation valves be closed. Since closure of the valves disables the RCIC system, other sections of the tech specs limit the length of time that the plant can be operated. Consequently, it was necessary to quickly diagnose and correct the situation.

Figure 1 illustrates the flow measuring instrumentation used. Each flow measuring instrument consists of the following elements:

1. *Flow elbow:* This is an ordinary 90-deg pipe elbow with taps at 45 deg, one on the outside and one on the inside of the elbow. The elbow may be mounted in the horizontal or

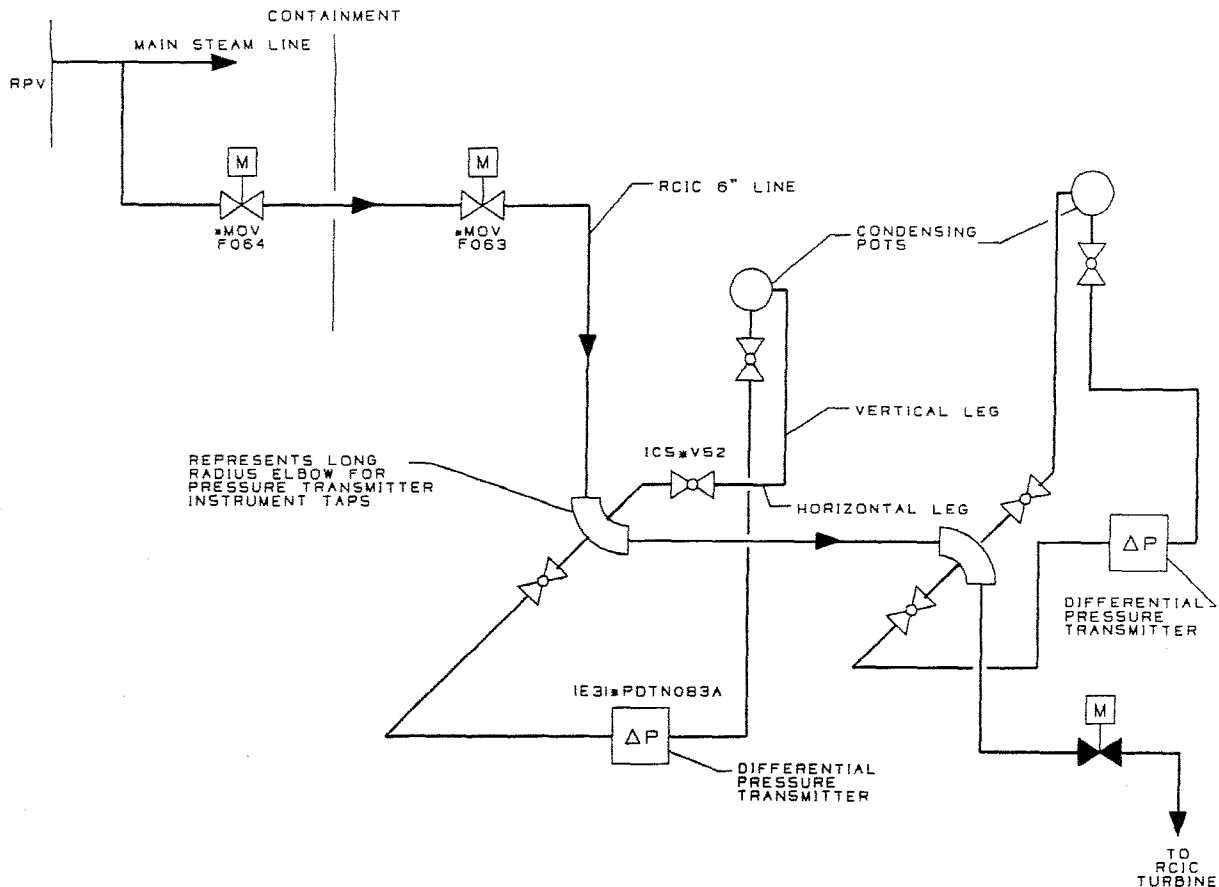


Fig. 1. Sketch of piping layout for Rosemount transmitters.

vertical plane. Steam flow through the elbow in either direction causes a differential pressure that is a function of flow.

2. *Condensing pot:* These are generally used for the top tap of a vertically mounted elbow or for both taps of a horizontally mounted element to ensure the instrument line is maintained full of water. In one case at RBS, condensing pots were used to gain enough elevation to permit proper sloping of the instrument line to the differential pressure transmitter.

3. *Differential pressure transmitter:* Converts the differential pressure to an electrical signal.

4. *Sensing lines:* This is small-diameter piping connecting the flow elbow, the condensing pot, and differential pressure transmitter. The piping from the elbow must be sloped upward to the condensing pot and permit a countercurrent flow of steam and condensate (i.e., flow of steam to the condensing pot and flow of condensate back into the steam line). The line from the condensing pot (or elbow if no pot is used) should be sloped continuously down to the transmitter so that no air pockets will form that may cause an erroneous reading.

The possible causes of the observed instrument line anomalies are as follows:

1. air in the sensing lines causing water legs, which biases the indication
2. incorrect calibration of the transmitter, including the wrong elevation of the condensing pot or elbow
3. faulty transmitter

4. the line between the flow elbow and the condensing pot partially or completely filling with condensate.

After a thorough investigation, items 1 through 3 above were ruled out, and item 4 was determined to be the cause. The reason that the line fills with water is that, at some point in the line, water accumulates and blocks steam flow to the condensing pot. When this happens, steam above the blockage gradually condenses and draws condensate up into the piping leading to the condensing pot. Factors that cause blockage are:

1. *Inadequate line slope:* The line slope should be a minimum of 0.5 in. per foot in 0.75-in. piping to ensure proper draining. It is important to realize that line slope may change as the plant heats up to rated conditions so line slope should also be checked after heat up.

2. *Valves:* Globe valves in nearly horizontal lines may cause blockage (see Fig. 2) due to the internal construction of the globe valve. Valves should be eliminated or located in vertical runs of piping.

3. *Obstructions:* Foreign objects inadvertently left in the piping during construction could cause a blockage.

4. *Fitting geometry:* The geometry of the fittings welded to the flow elbow (particularly the lower tap on the vertically mounted flow elbow) may result in blockage (see Fig. 3).

In the first case mentioned where the instrument indicated a substantial flow when no flow was present, it was postulated that the low-side leg was filling with water since this would explain the positive error observed. This leg has a globe valve installed in a nearly horizontal run of piping and was the likely

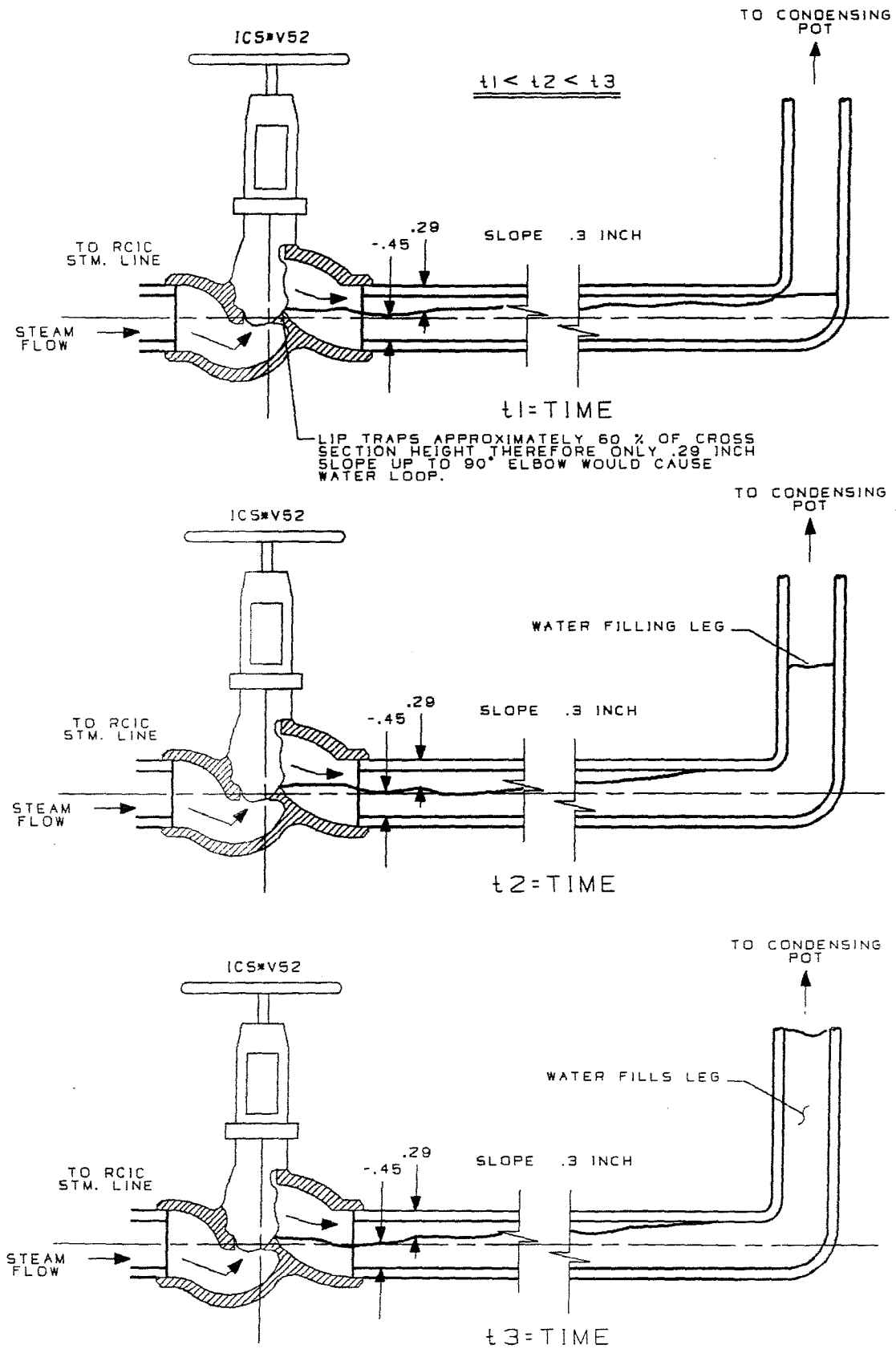


Fig. 2. Filling vertical leg on I-ICS-750-90-2(Z-).

- t1- FLOW BEGINS IN RCIC STEAM LINE CAUSING THE WATER LOCK IN THE FIRST VERTICAL 90° ELBOW.
- t2- CONDENSING STEAM CAUSES A LOWER PRESSURE IN THE VERTICAL LEG THEREBY DRAWING THE WATER UP THE PIPE AND FILLING THE LEG.

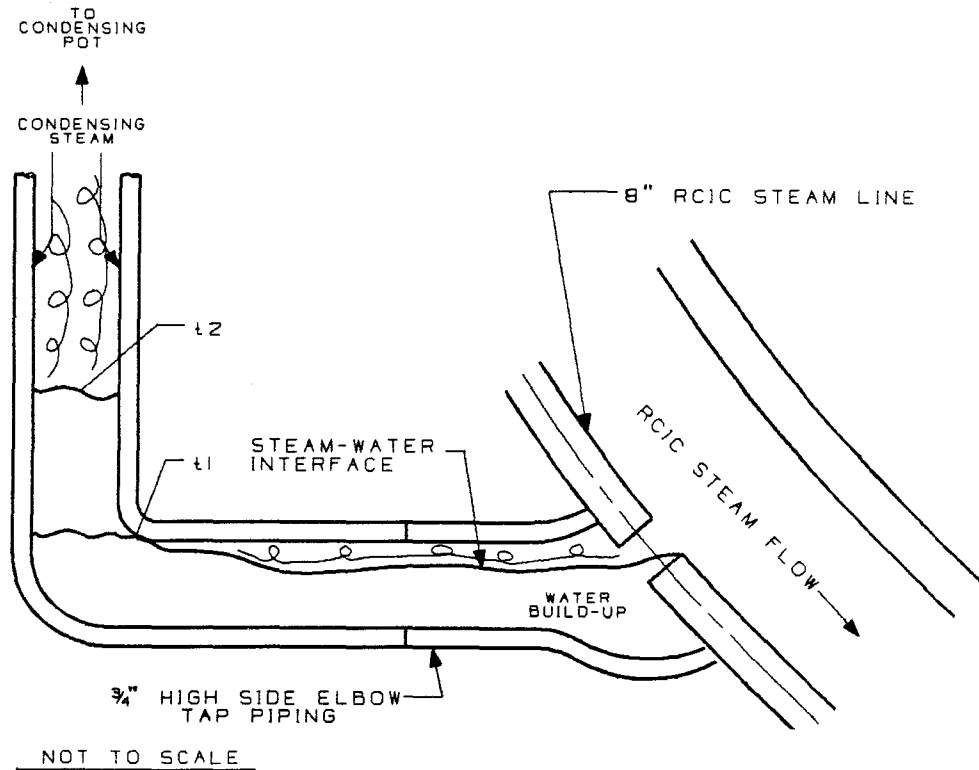


Fig. 3. Water pocket forming in sensing line.

cause. Subsequently, a design modification to this piping configuration (see Fig. 1), which moved globe valve ICS*V52 from the horizontal to the vertical leg, proved to be the solution. This modification prevented water from filling the vertical leg, thereby allowing the differential pressure transmitter to indicate properly.

Similar anomalies with the same instrumentation have been reported or observed at other plants in construction or initial startup. In one case, level instrumentation for the turbine moisture-separator-reheater drain tanks was observed to be installed incorrectly, having a globe valve in a nearly horizontal line, and using small diameter tubing from the tank to the condensing pot. Reference 1 describes a condition at a foreign pressurized water reactor in which accumulation of noncondensable gases in a pressurizer level instrument condensing pot resulted in erroneous indications. Thus, it is concluded that improper installation of condensing pots may be a generic problem that warrants special engineering attention to avoid plant startup delays.

RECOMMENDATIONS

Plant owners should inspect instrumentation that employ condensing pots to ascertain if the following criteria are satisfied by the piping connecting the process and condensing pot:

1. minimum line size of 0.75-in. pipe.
 2. minimum slope of 0.5 in. per foot (Double check the thermal expansion calculations of the sensing line to ensure that the slope does not change significantly during pipe line heatup.)
 3. no globe valves or, if present, installed in vertical runs
 4. line insulated up to condensing pot.
1. OLLE NOCKERT, "Calculations and Measurements of Condensation in Level Pots," Nuclear Network Information, Ringhals Nuclear Power Station (1983).